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INFORMAL REPORT

OF TRANSURANIC WASTES IN THE SUBSURFACE DISPOSAL AREA OF THE RADIOACTIVE WASTE MANAGEMENT COMPLEX AT INEL A BRIEF ANALYSIS AND DESCRIPTION

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A BRIEF ANALYSIS AND DESCRIPTION OF TRANSURANIC WASTES IN THE SUBSURFACE DISPOSAL AREA OF THE RADIOACTIVE WASTE MANAGEMENT COMPLEX AT INEL

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ABSTRACT

This document presents a brief summary of the wastes and waste types disposed of in the transuranic contaminated portions of the Subsurface Disposal Area during the period 1954 through 1970. Wastes included in this summary are organics, inorganics, metals, radionuclides, and atypical wastes. In addition to summarizing amounts of wastes disposed and describing the wastes, the document also provides information on disposal pit and trench dimensions and contaminated soil volumes. The report also points out discrepancies that exist in available documentation regarding waste and soil volumes and makes recommendations for future efforts at waste characterization.

SUMMARY

This document presents a brief summary of the wastes and waste types disposed of in the Subsurface Disposal Area (SDA) at the Radioactive Waste Management Complex. The information presented in this report was compiled through the review of existing documents and is limited to the transuranic waste containing portions of the SDA, Pits 1 through 6, 9, and 10 and trenches 1 through 10. In addition to describing the wastes and disposal locations, several discrepancies among currently available documents are pointed out, and recommendations for future efforts aimed at characterizing the wastes are made.

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Summary of contaminants and environmental media of concern	Estimates of metal content in selected pits and trenches at the SDA	Volumes of organic wastes shipped from RFP to the INEL 21	Special waste considerations buried in pits at the SDA \ldots 20	Waste container types, numbers, and total volumes for selected trenches at the SDA	Waste container types, numbers, and total volumes for selected pits at the SDA	Weight and volume fractions of wastes stored at the RWMC (Edinborough, 1990)	Waste Constituants Disposed in Rocky Flats TRU pits and Trenches at the SDA	Summary listing of makeup of waste buried in the SDA at RWMC	Clay mineralogy of selected surficial sediment samples 10	Mineralogy for subpit samples 9	Particle size distribution for subpit samples 9	Properties of soil and sediment samples from the RWMC wells	Various physical characteristics of soil and sediment samples from the RWMC wells

A BRIEF ANALYSIS AND DESCRIPTION OF TRANSURANIC WASTES IN THE SUBSURFACE DISPOSAL AREA OF THE RADIOACTIVE WASTE MANAGEMENT COMPLEX AT INEL

PURPOSE

The purpose of this report is to provide a brief overview and summary of the waste forms, types, and amounts buried in the Subsurface Disposal Area (SDA) at the Radioactive Waste Management Complex (RWMC) from 1952 to 1970. This document was prepared for use as necessary by programs within the EG&G Idaho, Inc., Waste Technology Development Department, other EG&G Idaho organizations, and outside organizations interested in demonstrating potential remedial technologies at the SDA.

HISTORY

Engineering Laboratory); wastes were later received from a variety of sources, were buried in a series of pits and trenches located in the area now known as (TRU) contaminated wastes (Barnes et al., 1989)ª, and low-level wastes (LLW)^b The RWMC was established in 1952 for the disposal of wastes generated 1952 to 1970 wastes consisting of solid wastes (Vigil, 1989), transuranic including government agencies, universities, and research laboratories. the National Reactor Testing Station (NRTS, now known as Idaho National

Currently, pit disposal at the SDA is utilized for LLWs only; trenches Originally, these wastes were not segregated at the time of disposal; contaminated waste was initiated at the Transuranic Storage Area (TSA) at generally placed in the trenches. In 1970, above ground storage of TRU however, TRU wastes were generally placed in the pits, while LLWs were are no longer used for waste disposal

years, For the purpose of this report TRU wastes are considered to be those wastes that are contaminated with alpha-emitting radionuclides that are heavier that uranium (atomic weight 92), have half-lives longer than 20 and are in concentrations greater than 10 nanocuries per gram.

b. LLW is defined as waste not classified as high-level radioactive waste, transuramic waste, spent nuclear fuel, or byproduct material as defined in section lle. (2) of the Atomic Energy Act (uranium or thorium tailings or

DESCRIPTION

TRU wastes were disposed at the SDA in Pits 1 through 6, 9, 10, 11, and 12 and in Trenches 1 through 10 (refer to Figure I for details) and in limited amounts in Trenches 16 through 54 (Horton, 1988). During earlier retrieval efforts (circa, 1978) the drummed wastes in pits 11 and 12 were removed (McKinley and McKinney, 1978); however, some wastes contained in boxes were left in the pits due to the deteriorated condition of the boxes (Horton, 1988). Pits 7 and 8 were used for the disposal of non-TRU wastes only (Card, 1977). Trenches were generally excavated to bedrock (basalt), approximately 10 ft down and averaged about 7 ft wide and up to 1800 ft long. Pits were excavated to bedrock and generally backfilled with 2 to 5 ft of soil to provide a level floor (Guay, 1989). Surface areas and volumes of the pits varied widely.

Following excavation, wastes were deposited into the pits and trenches. From 1952 until 1963, the waste containers (mainly steel drums and wooden and cardboard boxes) were stacked to optimize disposal space. From 1963 until 1969, the wastes were randomly dumped into the pits and trenches in order to limit worker radiation exposure. Beginning in 1969, the wastes containers were stacked in order to optimize disposal volume. Following emplacement of the wastes, the pits and trenches were backfilled and covered with at least 3 ft of the silty clay soil (Guay, 1989).

Excavated volumes of the pits and trenches are given in Table 1 (Guay, 1989). This table also includes the estimated volumes of soils placed in the pits and trenches as intermediate cover during placement of the wastes, backfilling of the pits and trenches, overburden placed after closure, and, in the case of the pits, underburden beneath the pits. Volumes of wastes disposed in the pits and trenches are also given. Excavation details of the individual pits and trenches are given in Tables 2 and 3.

The soils at the RWMC consist mainly of silty clays and sands. Details of the characteristics of INEL soils are presented in Tables 4 through 8.

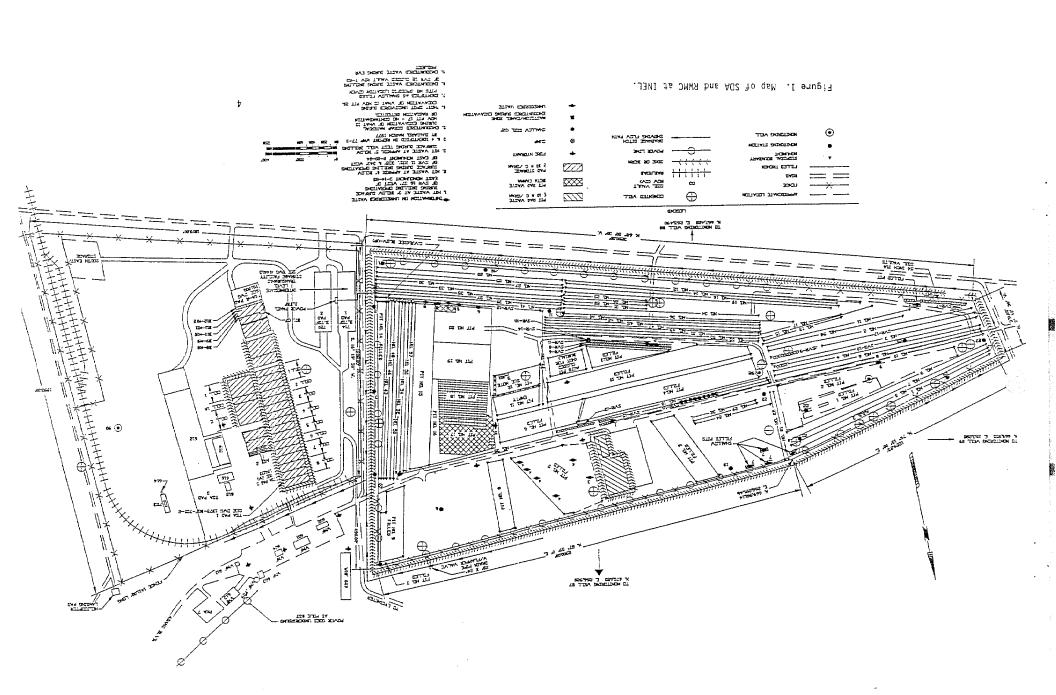


Table 1. Summary of volumes of pits, trenches, wastes, and associated soils in selected pits and trenches at the SDA (Guay, 1989)

Location	Excavated Volume ft3	Waste Container Volume ft3	Soil Volume ft3	Overburden Volume ft3	Subsidence Volume ft3	Underburden Volume ft3
T 1	81243	16897	64346	30563	0	NA
T 2	86932	6801	80131	26544	0	NA
T 3	90658	12375	78284	26664	689	NA
\mathbf{T} 4	93828	17788	76040	26808	750	NA
T 5	112362	17905	94457	29245	0	NA
T 6	91982	1 5475	76507	26856	1800	NA
T 7	87278	10729	76549	29093	0	NA
8 T	97752	14143	83610	26880	156	NA
T 9	83633	13237	70396	28891	0	NA
T 10	91474	9107	82368	26904	923	АИ
P 1	379135	81819	297316	107884	0	169532
P 2	1020359	418357	602002	425975	455	544852
P 3	368394	10205 9	266335	236150	0	70845
P 4	955309	388494	566815	787343	0	367427
P 5	796729	286612	510117	368236	18	100428
P 6	447515	223898	223617	409313	0	191013
P 9	342416	150690	191726	256812	353	149807
P 10	1052941	538865	514076	784084	0	526471
TOTAL	6279940	2325251	3954692	3654245	5144	2120375

a. Total volume of pits and trenches = $6,279,940 \text{ ft}^3$ Total volume of waste containers = $2,325,251 \text{ ft}^3$ Total volume of contaminated soil = $9,734,456 \text{ ft}^3$ Total volume of contaminated wastes and soils = $12,059,707 \text{ ft}^3$

Does not include wastes retrieved during early and interim waste retrieval projects.

Table 2. Excavation details for selected pits at the SDA (Guay, 1989)

Pit #	Mean Surface Elevation (ft)	Mean Basal Surface Elevation (ft)	Average Estimated Overburden (ft)	Estimated Excavated Depth (ft)	Estimated Width (ft)	Estimated Length (ft)	Surface Area (ft ²)	Excavated Volume (ft ³)
1	5016.3	4995	3.5 (5.5) ^a	12.0	68	455	30,824	379,135
2	5014.1	4994	4.3 (5.5) ^a	10.3	87	1150	99,064	1,020,359
3	5014.3	5000	5.0 (1.5) ^a	7.8	100	472	47,230	368,394
4	5013.1	4993	7.5 (3.5) ^a	9.1	104	1000	104,979	955,309
5	5011.9	4993	5.5 (1.5) ^a	11.9	150	450	66,952	796,729
6	5012.2	4993	7.5 (3.5) ^a	8.2	120	455	54,575	447,515
9	5010.5	4993	6.0 (3.5) ^a	8.0	130	330	43,802	342,416
10	5104.1	4993	7.0 (4.7) ^a	9.4	120	940	112,015	1,052,941

a. The value in brackets is the depth of the soil left in place over the basalt when the pits were excavated.

σ

Table 3. Excavation details for selected trenches at the SDA (Guay, 1989)

Trench #	Mean Surface Elevation (ft) 5,012.9	Mean Basalt Surface Elevation (ft) 4,997	Average Estimated Overburden (ft) 3.8	Estimated Excavated Deptha (ft) 10.1	Estimated Width (ft) 7	Estimated Length (ft) 1149	Surface Area (ft²) 8,043	Excavated Volume (ft ³) 81,243	· · · · · · · · · · · · · · · · · · ·
2	5,012.1	4,993	4.0	13.1	6	1106	6,636	86,932	
3	5,012.6	4,993	4.0	13.6	. 6	1111	6,666	90,658	
4	5,013. 0	4,993	4.0	14.0	6	1117	6,702	93,828	
5	5,013.6	4,997	3.8	14.6	6.6	1166	7,696	112,362	
6	5,012.7	4,993	4.0	13.7	6	1119	6,714	91,982	
7	5,014.2	4,997	3.8	11.4	6.6	1161	7,656	87,278	
8	5,013.1	4,993	4.0	14.1	6	1120	6,720	97,752	
9	5,013.8	4,997	3.8	11.0	6.6	1152	7,603	83,633	
10	5,012.6	4,993	4.0	13.6	6	1121	6,726	91,474	

a. The excavated depth was calculated by taking the difference between the surface elevation and the basalt elevation. From this value the estimated overburden was subtracted out. Finally, an additional 2 ft (1 ft for basalt overburden, 1 ft for depth of buried waste from original surface elevation) were subtracted out.

Table 4. Various Physical Characteristics of Soil and Sediment Samples from the RWMC Wells (EG&G Idaho 1989)

95	94	92	Well
10	6 6	2 6	Well Depth Interval Well Top Bottom No. (ft in.) (ft in.
12 6	ස ය	ហ	Depth Interval Top Bottom t in.) (ft in.)
2.66	2.67	2.65	Specific Gravity
1.70	2.02	1.87	Bulk Density (g/cm ³)
41.0	30.5	34.3	Porosity (percent)
13.2	16.4	12.9	Moisture Content (percent)
7.9×10^{-3}	2.7 x 10 ⁻⁴	5.5 x 10 ⁻⁴	Vertical Hydraulic Conduct (m/day)

Table ភ Properties of soil (EG&G Idaho, 1989) and sediment samples from the RWMC wells

Medianb	95	94	92	¥e11
	10	ອາ	2	Dept Top (ft
		o	2 6	<u>lepth Interval</u> Top Bottom (ft in) (ft in
	12	œ	5	terva Bott (ft
	On:	ω		in)
35.9 56.0	38.5	38.7	21.2	Partic Distri Clay
56.0	55.6	56.5	21.2 48.8 30.1	Particle Size Distribution (%) a Clay Silt Sand
7.3	5.9	4.8	30.1	Sand
2	1	ω	2	Clay Min Kaolin- ite
7	4	9	5	Clay Minerals (%) Moisture oolin- Illite Mon ite illi
თ	w	4.	5	Montmor- illonite
21	17	23	14	Cation Exchange Capacity (meg/100g)

œ. Clay <0.004 mm Silt 0.004 - 0.062 mm Sand 0.062 - <2.00 mm. Median of eight samples.

ŗ

of size distribution for subpit samples (in percent sample^a) (EG&G Idaho, 1989) Particle analyzed ů, Table

Sand Coarse ^c 0.5-1mm	0.1	0	0.1	0.1	
Sand Medium 0.25- 0.5 mm	6.0	0.2	0.2	0	
Sand Fine 0.125- 0.25 mm	2.3	17.7	0.7	9.0	
Sand Very Fine 0.0625- 0.125 mm	4.8	11.0	4.1	6.0	
Silt 0.004- 0.0625 mm	38.5	30.1	40.1	8.69	
Clay <0.004 nun	53.3	41.0	54.7	23.5	
Depth (in meters)	0.91	1.22	1.52	1.83	
Sample ^b Number (EWR-1-4	EWR-1-3	EWR-1-2	EWR-1-1	

Analyzed by the USGS Hydrologic Laboratory, Denver, Colorado.

٠. ت All samples from surficial deposits.

c. No particles coarser than 1 mm observed.

Mineralogy for subpit samples (in percent of analyzed sample)^a (EG&G Idaho, 1989) . Table

Total Percent	113+6	103+	87+	+66
Clay Minerals	70	55	25	40
Pryoxene Dioxide	4	6	6 >	6√
Calcite	1	0	41	13
Potassium Feldspar Plagioclase Calcite	11	10	9	12
Potassium Feldspar	5≥	9₹	Ø	យ
Quartz	27	53	15	59
Sample ^b Depth Number (in meters) Quartz (0.91	1.22	1.52	1.83
Sample ^b Number	EWR-1-4	EWR-1-3	EWR-1-2	EWR-1-1

the USGS Hydrologic Laboratory, Denver, Colorado. Analyzed by ٠ ا

b. All samples from surficial deposits.

c. Due to high percentage of clay minerals.

Clay mineralogy of selected surficial sediment samples (in percent of total clay minerals/percent of original bulk samples) (EG&G Idaho, 1989) Table 8

ate nt s}				
Carbonate Content (CaCo ₃) Percent	0	2.3	36.1	10.8
Cation (Exchange Capacity P	27	27	11	11
Mixed Layer Clays (Illite/ Smectite) Smectite Kaolinite	6/4	6/3	9/2	12/5
Smectite	13/9	15/9	24/6	26/10
Mixed Layer Clays (Illite/ Smectite)	45/32	48/26	31/8	30/12
l Illite	6/25	0/16	6/9	2/12
Chlorite	0/03	0/03	6/03	0/03
Depth Sample (in meters) Chlorite Illite	EWR-1-4 0.91	1.22	1.52	1.83
Sample	EWR-1-4	EWR-1-3	EWR-1-2	EWR-1-1

Analyzed by the USGS Hydrologic Laboratory, Denver, Colorado. In milliequivalents per 100 g. ъ.

WASTES DISPOSED

During the period 1954 to 1970 a wide variety of wastes and waste types were shipped to the SDA for disposal from on- and off-site generators. Table 9 presents a summary description of the makeup of the wastes buried at the SDA. Wastes of concern disposed of at the SDA include organics, inorganics, toxic metals, and radionuclides. Table 10 presents a listing of typical waste contaminants found at the SDA. A summary of the waste types buried at the SDA, giving weights and volumes, is presented in Table 11.

A variety of containers were utilized for the shipment of wastes to the SDA. These containers included steel drums (30, 40, and 55-gal), cardboard cartons, and wooden boxes (up to 105 in. x 105 in. x 214 in.). Larger individual items were disposed separately as loose trash. In addition, large amounts of plastic were used to line the containers and to wrap some of the boxes and most of the larger individual items. These plastics included polyethylene sheet plastic and drum liners, polyvinyl chloride sheets and liners, and plastic jugs and other containers of unknown composition. Tables 12 and 13 present a breakdown of the number of containers disposed of in the pits and trenches, respectively, at the SDA during the period 1954 to 1970. Waste items that may present a particular challenge for remedial technologies are summarized by pit in Table 14.

Nonradiological wastes of concern include primarily hazardous organics and various metals. Much of the organics were shipped to the SDA from off-site sources, particularly the Rocky Flats Plant (RFP). An estimate of the amounts of various organic compounds shipped from RFP to the SDA is given in Table 15. Estimates of the amount of organics generated at INEL and disposed in the SDA are not available. Estimates of the amounts of metal disposed at the SDA have generally included only nontoxic metals. These metals are of concern due to the large amounts present in the pits and trenches at the SDA and the difficulties that they may present during remediation of the wastes contained at the SDA. Estimated metal content is presented by pit/trench in Table 16.

Summary listing of makeup of waste buried in the SDA at RWMC (McKinley, 1978) Q Table

Lumber, wallboard, concrete blocks, steel plate and shapes, ducting, electrical wires, fuse boxes, roofing material, floor tile, insulation, lead sheet and bricks, asphalt paving material, soil, sand, gravel, steel stairways, and ladders.	Hoods, laboratory benches, desks, chairs, cabinets, glassware, plastic tubing, plastic and glass bottles, solutions stabilized in concrete or plaster, and vermiculite.	Tanks, heat exchangers, tube bundles, condensers, pumps, piping, flanges, valves, organic wastes, ion exchange resins, zirconium plate, zirconium turnings, sawdust, and HEPA filters.	Hand tools, metal-working machines, cranes, hoists, welders, oils and grease, metal filings, and abrasive wheels and sheet.	Paper, rags, plastic bags and sheet, floor sweepings, brooms, and steel wool.	Sewer sludge, garbage, animal remains and excreta, jet engines, vehicles, Test Reactor Area fuel end boxes.
Construction and Demolition Material	Laboratory Equipment and Materials	Process Equipment	Maintenance Equipment	Decontamination Materials	Miscellaneous

a. This listing is for the entire Subsurface Disposal Area (SDA). For the purpose of this document it is considered to be representative of the makeup of the TRU-contaminated waste.

Waste Constituants Disposed in Rocky Flats TRU Pits and Trenches at the SDA Table 10.

Radionuclides

Chromium (unknown isotopes) Mixed activation products Unidentified beta/gamma fission products Ruthenium-Rhodium-106 isotopes) Cerium-144 (as CeCl₃) Cesium-137 Aluminum isotopes Americium-241 Nickel-63 Phosphorus-32 Plutonium-238 Plutonium-239 Plutonium-240 Plutonium-241 Plutonium-242 Promethium-147 Strontium-90 Sulfur-35 Thallium-204 Polonium-210 Chlorine-36 Chromium-51 Thorium-232 Thulium-170 Uranium-234 Uranium-235 Uranium-238 Iron (other Selenium-75 Californium Iridium-192 Uranium-233 Radium-226 Calcium-45 Iodine-131 Yttrium-91 Neptunium Sodium-22 Carbon-14 Cobalt-60 Iron-55 Iron-59 Tritium Zinc-65 Curium Mixed

Metals

Aluminum
Beryllium
Carbon steel
Copper
Iron
Lead
Lithium
Magnesium
Mercury
Molybdenum
Platinum
Potassium (as NaK)
Sodium (metal and NaK)
Stainless steel
Tantalum
Tin
Yttrium (as Y₂O₃)
Zircaloy
Zirconium

Inorganics

d magnesium oxides d plutonium oxides d silicon oxides Uranium-zirconium hydride americium oxides aluminum oxides iron oxides Euxenite ore residue Hydrochloric acid Hydrofluoric acid Asbestos Beryllium oxide Calcium silicate Chrome brick Lithium hydride Nitric acid Perchloric acid Portland cement Sulfuric acid Thallium oxide Zirconia brick Alumina brick Hydrofluoric Iron oxides Argon (gas) Hydrated Hydrated Hydrated Hydrated Hydrated Hydrated Graphite Silica Lime

Salts

Barium carbonate Barium fluoride Iron chlorides Potassium nitrate Sodium chloride Sodium nitrate Thorium fluoride Uranium fluoride Radium salts

Organics

Diisopropyl fluorophosphate Dimethyl (1,4 bis-2,5 phenyloxazolyl benzene) Diphenyls Sodium isobutyrate Texaco Regal oil Terphenyls (ortho, meta, and para) Polyethylene Polychlorinated biphenyls Polyvinyl chloride Polyurethane foam Ethylene glycol Formalin and formaldehyde 1,1,1-trichloroethane Trichloroethylene Butyl stearate Carbon tetrachloride 3-methylcholanthrene **Perchloroethylene** Acetic acid Acetic anhydride Organophosphates Nitrocellulose Methocel 400 Methyl alcohol 400 Nitrobenzene Anthracene Cellulose Gasoline Benzene Beeswax Parafin [o] uene Freon

VERSENE (ethylenediaminetetraacetic acid)

Miscellaneous oils:
Gearbox oil
Hydraulic oil
Machining oils
Spindle oil
Unidentified:
acids
alcohols
amino acids
esters
insecticides
plastics
proteins
pyrimidines
solvents

Weight and volume fractions of wastes stored at the RWMC (Edinborough, 1990) Table 11.

Category	Codes	Weight Fraction ^a	Volume Fraction ^a	Density (1b/ft³)
Combustibles	40	0.201	0.42	17.73
Sludge	36	0.327	0.18	67.22
Unknown/unclassifiable	21	0.097	0.15	24.87
Metals	15	0.222	0.086	95.87
Mixed waste	22	0.0389	0.073	19.86
Concrete, brick, particulates	38	0.0791	0.045	65.70
Nonmetals and glass	18	0.0284	0.041	25.95
Low level	1	0.00389	0.005	31.81
Remote handled	2	0.000961	0.001	25.31
NonTRU	~ 1	0.00133	0.001	47.87
Salts	11	0.000691	0.0007	39.37

and volume fractions of the buried TRU wastes are assumed to be given for the stored TRU wastes. Weight as that а. Ѕате

Table 12. Waste container types, numbers, and total volumes for selected pits at the SDA (Guay, 1989)

10	Q	თ	ហ	45	ω	2	 4	Pit No.
Drums	Drums	Drums	Drums	Drums	Drums	Drums	Drums	Container Type
Wooden Boxes	Wooden Boxes	Wooden Boxes	Wooden Boxes	Wooden Boxes	Wooden Boxes	Wooden Boxes	Wooden Boxes	
Cardboard Boxes	Cardboard Boxes	Cardboard Boxes	Cardboard Boxes	Cardboard Boxes	Cardboard Boxes	Cardboard Boxes	Cardboard Boxes	
Other	Other	Other	Other	Other	Other	Others	Other	
27,101 2,311 914 295 30,621	3,937 520 1,932 72 6,461	13,912 590 3,523 36 18,061	19,652 919 970 102 21,643	31,467 624 2,020 268 34,379	6,684 201 3,309 62 10,256	34,480 1,048 3,547 443 39,518	8,285 152 2,173 2,0,612	No. of Containers
189,857	28,942	102,272	144,355	231,330	48,961	252,077	60,917	Volume of Containers(ft ³)
274,048	72,735	73,918	110,831	68,060	10,565	75,728	8,001	
11,830	29,571	41,242	7,773	16,617	30,774	17,960	12,869	
63,130	19,442	6,466	23,653	72,487	11,759	72,592	32	
538,865	150,690	223,898	286,612	388,494	102,059	418,357	81,819	

Table 13. Waste container types, numbers, and total volumes for selected trenches at the SDA (Guay, 1989)

10	ဖ	œ	7	თ	ហ	4	ω	2	L	Trench No. a
Steel drums Cardboard boxes	Drums Wooden boxes Cardboard box	Drums Cardboard boxes	Drums	Drums Wooden boxes	Drums	Drums Wooden Boxes	Drums Wooden Boxes Cardboard Boxes	Drums Wooden Boxes	Drums Other	Container Type
1,236 7 1,243	1,769 1 2 1,772	1,654 793 2,447	$\frac{1,497}{1,497}$	2,283 1 2,284	2,541 2,541	2,416 1 2,417	1,242 6 1,423 2,671	1,045 4 1,049	3,376 1 3,377	No. of Containers
9,089 35 9,124	13,008 224 10 13,242	12,160 3,965 16,125	10,729 10,729	15,462 13 15,475	18,176 18,176	17,761 27 17,788	8,655 162 7,115 15,932	6,761 40 6,801	16,747 150 16,897	Volume of Containers (ft ³)

^{&#}x27;n The stored waste in Trenches 1-10 consisted mainly of cardboard boxes from the INEL on-site generators. Intermixed with the on-site boxes were steel drums, wooden boxes, plastic bags, and loose waste. Some Rocky Flats waste is also intermixed. Values reflect the retrieved waste removed.

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Special waste considerations buried in pits at the SDA (Card, 1977) Table 14.

Pit No.	Comments
 1	Pit reopened in October, 1961, for disposal of some waste from the SL-1 reactor incident. Waste contained approximately 120 Curies of MFP. Highest radiation reading recorded was 10 R/h at contact. This waste is probably limited to the western end of the pit.
2	Large amounts of beta-gamma contaminated waste disposed with the TRU drums and boxes. This waste includes reactor shielding (36,000 lb), an aluminum heat exchanger (20,000 lb), drums with various contents, large (150 ft ³) concrete blocks contaminated with MFP, and other items. Also this pit contains contaminated material from the cleanup of the SL-1 incident. This pit was flooded in 1962, prior to closure.
က	NonTRU boxes from various on-site sources buried in this pit. These boxes apparently contain MFP. Six boxes (12,000 to 14,000 lb each) of intermediate-level waste (55-gal drum centered in a box, and the box was then filled with concrete shielding) buried at unknown locations in pit. SL-1 incident cleanup waste also buried in this pit.
4	MFP wastes intermixed with TRU wastes; locations are not clearly recorded (probably randomly dumped into the pit). Non-TRU waste (containers, scrap lumber, concrete, miscellaneous equipment, scrap metal, valves, and piping) generally confined to the westernmost 91 m of the pit.
ဟ	Burial location for some of the wastes may be in error. A good possibility exists that some of the wastes may actually be outside of recorded pit boundaries.
9	Drums and boxes were not segregated during burial. A large number of boxes containing contaminated empty drums are buried in this pit.
6	Drums and boxes were not segregated. A large number of boxes containing contaminated empty drums are buried in pit.
10	Contains large number of boxes of contaminated empty drums, but the number of drums per box is not known. This pit was flooded in the spring of 1969, before it was closed.

1987) (Kudera, Volumes of organic wastes shipped from RFP to INEL Table 15.

lons)	Other Organic ^a	Rec 4,2 8,3	24,968 35,135 16,564 10,771	6,846 5,356 5,371	4,743 12,431 6,398 7,925 10,127	7,6/0 9,240 120 142,876	
(Volumes in Gallons	Carbon Tetrachloride	^ ^ 4		5,862 6,106 4,304 3,032	2,531 2,930 3,930 7,056	4,491 6,792 5,764 5,744 3,235 83,226	
	Texaco Regal Oil	0,4,40	39,018 2,528 8,911 11,322	8,794 9,158 6,456 748	3,796 3,796 5,096 8,557	6,736 10,189 8,646 8,852 8,617 4,853 124,842	•
	Number of Orums	~ ~	8,709 1,092 825 953	596 572 539 361	334 471 482 672 674	480 387 300 463 155 9,284	
	RWMC Status		Total Buried Stored Stored Stored	Stored Stored Stored	Stored Stored Stored Stored	Stored Stored Stored Stored Stored Stored	.
	Year	1966 1967 1968 1969		1973 1974 1975	1977 1978 1979 1980	70	

Mostly 1,1,1-trichloroethane, trichloroethylene, perchloroethylene, and used oil.

b. Data estimated.

In 1984, this category of organic wastes were processed separately and shipped to the Nevada Test Site as low-level wastes. ن

d. January to June 1987.

Table 16. Estimates of metal content in selected pits and trenches at the SDA (Garcia et al, 1989)

TOTAL	Trench 10	Trench 9	Trench 8	Trench 7	Trench 6	Trench 5	Trench 4	Trench 3	Trench 2	Trench 1	Pit 10	Pit 9	Pit 6	Pit 5	Pit 4	Pit 3	Pit 2	Pit 1	Waste Location
27,962,900	110,500	179,300	208,000	198,800	305,000	347,200	267,600	196,500	123,700	274,500	6,148,000	1,357,707	2,672,000	2,968,000	5,539,000	823,500	7,264,000	337,300	Total Weight of Waste (kg)
22,370,320	88,400	143,440	166,400	159,040	244,000	277,760	214,080	157,200	98,960	219,600	4,918,400	1,086,166	2,137,600	2,374,400	4,431,200	658,800	5,811,200	269,840	Maximum Metal Weight ^a (kg)
1,398,145	5,525	8,965	10,400	9,940	15,250	17,360	13,380	9,825	6,185	13,725	307,400	67,885	133,600	148,400	276,950	41,175	363,200	16,865	Minimum Metal ^a Weight (kg)

a. Maximum metal weight was assumed to be 80% (worst-case scenario) of the total weight of the waste. Minimum metal weight was assumed to be 5% of the total weight of the waste.

Radiological wastes of primary concern at the SDA include TRU wastes and LLW, which include mixed-fission products (MFP) and mixed activation products (MAP). In addition, some high radiation level wastes were disposed of in trenches at the SDA (Vigil, 1989). Currently, the TRU wastes are felt to be of most concern as a threat to human health and the environment. The amount of TRU nuclides originating from RFP buried in the SDA from 1954 to 1970 is 381.3 kg, corresponding to 241,531 Curies (Lee, 1971) (Garcia and Knight, 1989). In addition, 203,322.5 kg (74.7 Curies) of various uranium isotopes were also shipped from RFP to the SDA (Lee, 1971) (Garcia and Knight, 1989). Following the retrieval efforts in Pits II and I2, approximately II6 boxes of TRU wastes remain in these pits (Horton, 1988). Assuming an average size of 4 x 4 x 7 ft for boxes received from RFP, the corresponding TRU waste volume equals 17,359 ft3. According to Horton (1988), Trenches 16 through 54 contain a total of 4,367 ft^{3} of TRU wastes (potentially mixed with MFP) received from on-site waste generators. For LLW, a total of 583,000 Curies (which includes mixed fission and activation products) were buried in the SDA with the TRU wastes (Vigil, 1989); it is not clear, however, if this amount includes only INEL-generated wastes or both INEL and off-site generated LLW. (In addition, because TRU and LLW wastes were not completely segregated prior to 1970, TRU wastes may also be present in Trenches II through 15; this cannot, however, be substantiated without a review of the shipping records for these trenches.)

The disposal records include a variety of sludges having been disposed in the SDA. These sludges include process sludges from RFP (74 series sludges), sewage sludge, and miscellaneous other sludges. The greatest volume of sludges were those received from RFP (Vigil, 1989) and included the following.

741 sludge is a wet sludge produced from treating aqueous process
wastes, such as ion-exchange column effluents, distillates, and
caustic scrub solutions. The caustic scrub solutions contain
ferric sulfate, calcium chloride, magnesium sulfate, and
flocculating agents. These chemicals form a precipitate of the

hydrated oxides of iron, magnesium, aluminum, silicon, plutonium, and americium and may contain low concentrations of beryllium.

- 742 sludge is a wet sludge produced from the treatment of all other plant radioactive and/or chemical contaminated wastes and further treatment of 741 sludge. This type of sludge may also contain mercury, lithium batteries, and small amounts of contaminated mercury in pint bottles. The same treatment chemicals were used as in the 741 sludge and the same precipitates formed.
- 743 sludge was produced from treatment of liquid organic wastes. The sludge waste consists of such materials as degreasing agents (mostly trichloroethane), lathe coolant (60% Texaco Regal Oil and 40% carbon tetrachloride), and hydraulic oils. Other organic wastes included trichloroethylene, tetrachloroethylene, gearbox and spindle oils, and trace amounts of miscellaneous organic laboratory wastes (e.g., organophosphates and nitrobenzene). There are also some unknown volumes of oil containing PCBs that were processed with this type of sludge.
- 744 sludge resulted from processing liquid waste not compatible with the 741 and 742 processes due to their plutonium complexing nature. The complexing chemicals included some alcohols, organic acids, and VERSENES (trade name for chelating agents containing ethylenediaminetetraacetic acid). These were added to Portland cement to solidify the wastes.
- 745 sludge is a salt waste originating from evaporation of liquid waste impounded in solar evaporation ponds at the RFP. The salt is estimated to consist of 60% sodium nitrate, 30% potassium nitrate, and 10% miscellaneous material.

MIGRATION OF WASTES

In recent years, data have been gathered documenting the migration of some waste constituents away from the SDA (EG&G Idaho, 1989). In particular, several organic compounds (including carbon tetrachloride, chloroform, tetrachloroethylene, 1,1,1-trichloroethylene, and trichloroethylene) have been detected at elevated levels in the sedimentary interbeds beneath the RWMC and in wells near the SDA (Hodge et al, 1989). Radionuclides (including Pu-238, Pu-239, Pu-240, Am-241, Cs-137, and Sr-90) have been detected in the soils and in sedimentary interbeds to a depth of 240 ft beneath the RWMC (Hodge et al, 1989). Table 17 describes the migration of contaminants from the SDA into the surrounding media. Figures 2 and 3 present diagrams of the migration pathways for radionuclides and hazardous contaminants and for volatile organic chemical contaminants, respectively.

Table 17. Summary of Contaminants and Environmental Media of Concern (Hodge et al, 1989)

	Presence of									
Environmental Medium	Organics	Inorganics	Radionuclides							
Air Surface water	Х	Х	a							
Soi1	X	a a	χ a							
Bedrock Interbeds	8	X	X							
Groundwater	X	X	X							

a. Sampling results detected contaminant presence at levels below environmental concern, but worthy of note.

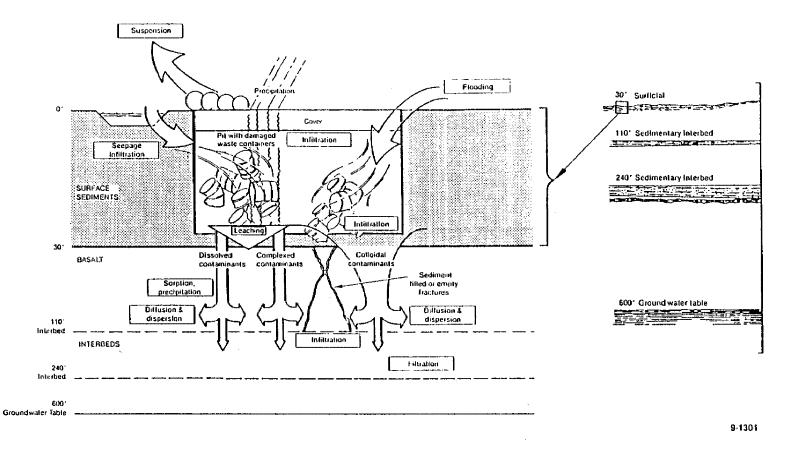


Figure 2. Conceptual model of radionuclide and hazardous contaminant migration at the SDA (EG&G Idaho, 1989).

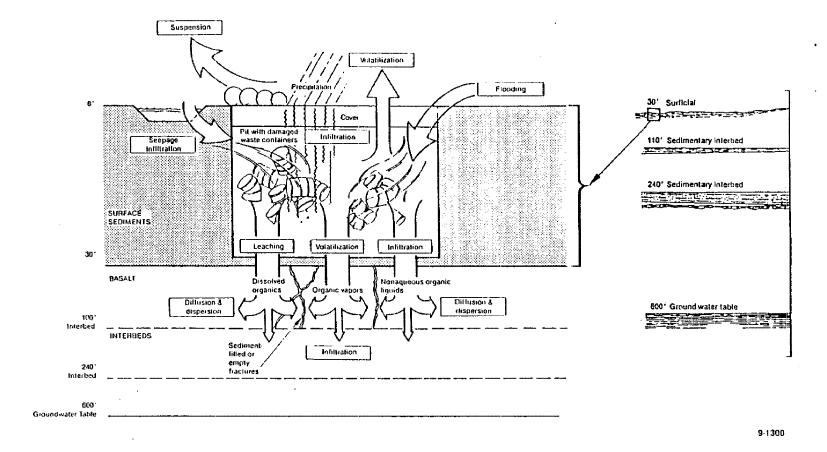


Figure 3. Conceptual model of volatile-organic chemical contaminant migration at the SDA (EG&G Idaho, 1989).

ASSUMPTIONS

In order to compile the summary information presented in this document, the following assumptions were made.

- Earlier retrieval efforts (Thompson, 1972) (McKinley and McKinney, 1978) (Bishoff and Hudson, 1979) discovered a large number of breached drums and decayed cardboard and wooden boxes. Because it has been 13 years since these efforts took place and because the wastes had been in place for less than 10 years at the time of these retrieval efforts, the majority of the waste containers are now assumed to have been breached or otherwise somewhat deteriorated. Degradation of the waste containers, however, may be offset somewhat by the internal plastic packaging, which may still retain its integrity.
- estimates, the contaminants are assumed to have leaked or migrated out into the surrounding soils. All underburden and interstitial soils associated with Pits 1 through 6, 9, and 10 and Trenches 1 through 10 are assumed to be TRU contaminated. As a result of discoveries made during the earlier retrieval operations (McKinley and McKinney, 1978) the top 18 in. of overburden in the pits and trenches are assumed to be uncontaminated; the remainder of the overburden, however, is assumed to be contaminated.
- Because these wastes were disposed during the period 1952 to 1970, the corresponding definition of TRU wastes was used (see footnote page 2).

- Although no records exist for waste shipments to the SDA originating from INEL (earlier known as the National Reactor Testing Station NRTS) sources prior to 1960 and because of the activities being performed at the different facilities (RFP and INEL), the vast majority of the TRU wastes are assumed to have originated off-site. Due to the types of operations that were conducted at the INEL, the majority of LLW is assumed to have come from on-site generators. The pits 1 through 6, 9, and 10 generally contain more RFP TRU waste than INEL LLW waste; the opposite is true for Trenches 1 through 10 (Vigil, 1989).
- TRU wastes were limited to the pits and trenches considered in this document. However, containers of waste may be located outside of established pit/trench boundaries, particularly near Pit 5 (Card, 1977) and Pit 2 (Thompson, 1972).
- Waste volumes given do not, with the exception of Pit 9 metal contents, include the volume of container/packaging materials. This is because volumes of metal drums, wood, and plastics may be considerable (wood was often used to shore bulky waste items in the wooden boxes during shipment to prevent movement during shipping, and most items were wrapped in at least a single layer of plastic prior to disposal). It is unclear if the volumes of wastes included materials (e.g., lead sheeting) used for shielding in the waste containers and whether these volumes are significant.
- Mixed wastes were not considered separately from the other organic and TRU wastes.
- The description and quantities of wastes disposed at the SDA are not expected to change significantly from those presented in this document following a more detailed review of the available literature and shipping records. This review is pending.

DISCREPANCIES

Although estimates of the combined volume of wastes and contaminated soils in the TRU waste pits and trenches at the SDA include 7.7 million ft³ (Plessinger, 1988) and 8.0 million ft³ (Humphrey and Bishoff, 1980), the degree of detail and method of estimating this volume provided by Guay (1989) lends greater credence to the estimate quoted in this document (e.g., 12.1 million ft³). The estimate of 8.2 million ft³ quoted by Plessinger (1988) and attributed to Humphrey and Bishoff appears to be in error. In addition, Plessinger's estimate does not include underburden volume; although Humphrey and Bishoff's estimates do include underburden, their calculations are not sufficiently documented to permit comparison. Guay's estimate includes not only overburden and underburden, but also presents a breakdown of volumes by pit and trench based on individual basalt and soil surface elevations and takes into account subsidence and compaction of waste containers in trenches. A large portion of these discrepancies may be due to additional overburden placed during subsequent SDA regrading operations.

Guay (1989) cites a number of references that give a total waste volume (from RFP, other off-site generators, and NRTS/INEL generators) buried at the SDA from 1952 through 1970 as 4.16 million ft³. In his calculations of waste container volume buried in the TRU contaminated portions of the SDA, Guay gives a total of 2.32 million ft³. Other estimates of total waste volume include: 2.67 million ft³ (Humphrey and Bishoff, 1980) (Vigil, 1989), 2.34 million ft³ (Garcia et al, 1989), 2.7 million ft³ (as both TRU and beta-gamma wastes) (Hinckley, 1981), and 2.3 million ft³ (McKinley, 1978). It is assumed that these numbers also represent wastes buried in the TRU contaminated portions of the SDA. Although some of these numbers may have since been discredited in subsequent efforts, they are provided to demonstrate the wide discrepancies that exist in estimates of waste amounts buried at the SDA. Obviously, further work is necessary to refine these numbers and explore this discrepancy.

Differences also exist in the estimated amounts of TRU radionuclides present at the SDA. ORNL (1989) presents a total mass of TRU nuclides (of DOE/defense origin) of 357 kg (as of 1988). Barnes et al (1989) cites the Pu²³⁹ content of the buried wastes as being 338 kg. Lee (1971) claims that 381.3 kg of TRU nuclides were shipped from the RFP to the SDA during the period 1954 to 1970; of this amount, 343.3 kg were Pu²³⁹. It is not clear whether the numbers cited by ORNL and Barnes et al include TRU nuclides generated on the INEL Site or by off-site generators other than RFP. The reduction in TRU nuclides resulting from decay during the intervening period (1970 to 1988/1989) may not account for the entire discrepancy, particularly if TRU wastes from generators other than the RFP were received at the SDA. Again, further work is necessitated to determine the amount of TRU nuclides present in buried wastes at the SDA. Lee (1971) has been cited in this report because it is not only the more conservative number, but also because this number was supplied directly from RFP, the major generator of TRU wastes.

This report was developed through the review of a large number of existing documents that present a great deal of information on the waste types, quantities, and locations. The information selected for inclusion in this report, however, was chosen because it appeared to be the most valid and defensible of the information contained in the reports reviewed in the time allowed. An example of this is Card (1977), which contains a series of diagrams of the TRU-containing pits showing general waste disposal locations; these diagrams were not included in this report. This is because the source of the information used to develop these drawings is not felt to be entirely accurate. Thus, this report represents what is felt to be the best available information at the time of release. As more information becomes known, and as more validity is afforded to the existing information not included in this report, this report will be revised.

PROPOSED FUTURE ACTIONS

In light of the questions arising from the assumptions made and discrepancies identified during preparation of this document, several additional efforts to reduce the uncertainties regarding buried wastes (including locations, contaminants, conditions, and quantities) are proposed to be conducted by the staff of the Technology Demonstration group at EG&G Idaho.

- Identify and obtain existing site characterization documents and reports from earlier retrieval efforts. This has, to a large extent, already been accomplished, and files containing this information have been set up.
- Identify current site characterization efforts within other organizations, such as Waste Area Group-7 (WAG-7). Although the extent of current site characterization efforts has not been determined, initial contacts with personnel involved with identified efforts have been made. In these cases, agreement has been reached regarding the need for a coordinated site characterization effort, such as that currently proposed.
- Review existing information to identify primary information sources, determine data quality, ensure consistency, and establish a valid foundation for future efforts.
- Compile waste amounts, waste types, and identification of waste disposal locations from the existing documents reviewed above.
- Review existing shipping records dating from the period of burial of RFP wastes at the SDA to determine types, dates, and volumes of wastes shipped from RFP to INEL. This effort is currently being performed by WAG-7; however, no coordination in this effort is apparent and the time frame for completion does not coincide with Waste Technology

Development Department milestones. We propose that these records be reviewed by personnel having strong technical backgrounds with the purpose of updating the existing Radioactive Waste Management Information System database and validating existing information on waste volumes and disposal locations.

- Emphasize attempts to verify the locations of "special case" wastes and to establish a confidence level with relation to the existing information sources. The intent will be to utilize current, state-ofthe-art geophysical techniques, in addition to the existing waste disposal records, to determine the locations of these special case wastes and to define the boundaries of the pits and trenches.
- Reconstruct, to the degree attainable, shipping records for on-site generators of wastes buried at the SDA. These records were destroyed during previous "housekeeping" efforts, but can be reconstructed to a limited extent from knowledge of the work processes and research activities generating the wastes. This effort has been begun by members of the Risk Assessment unit under Bob Nitschke, but is currently on hold pending budget allocations and anticipated workload.
- Estimate current radionuclide inventories in buried wastes at the SDA by applying rate of decay algorithms to the reviewed and verified radionuclide estimates resulting from the proposed effort.

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